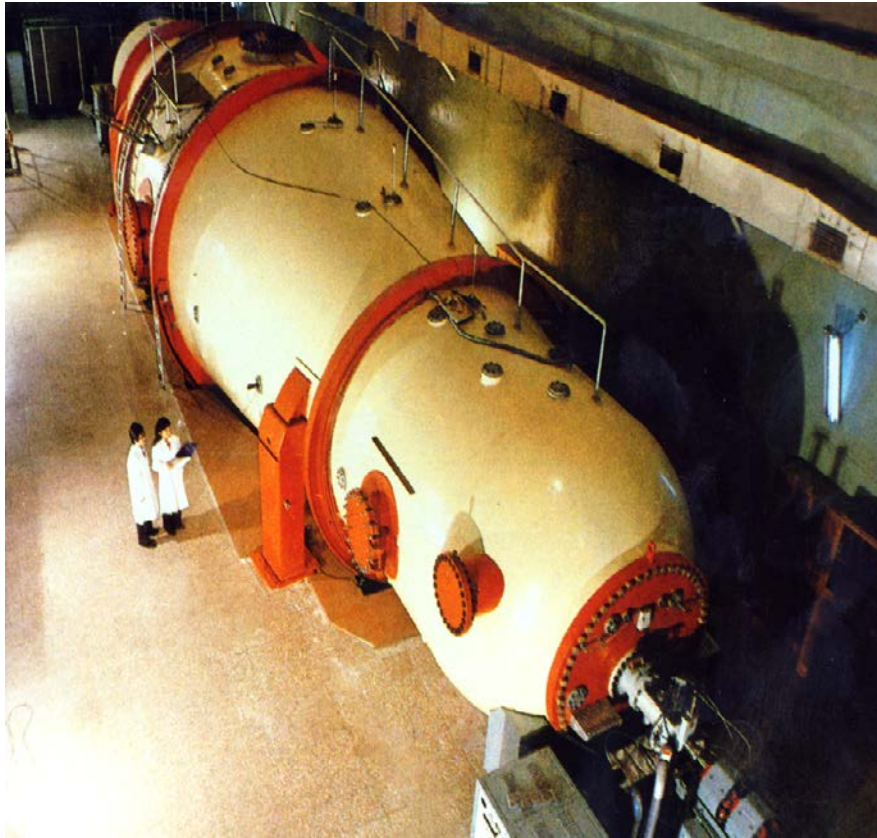


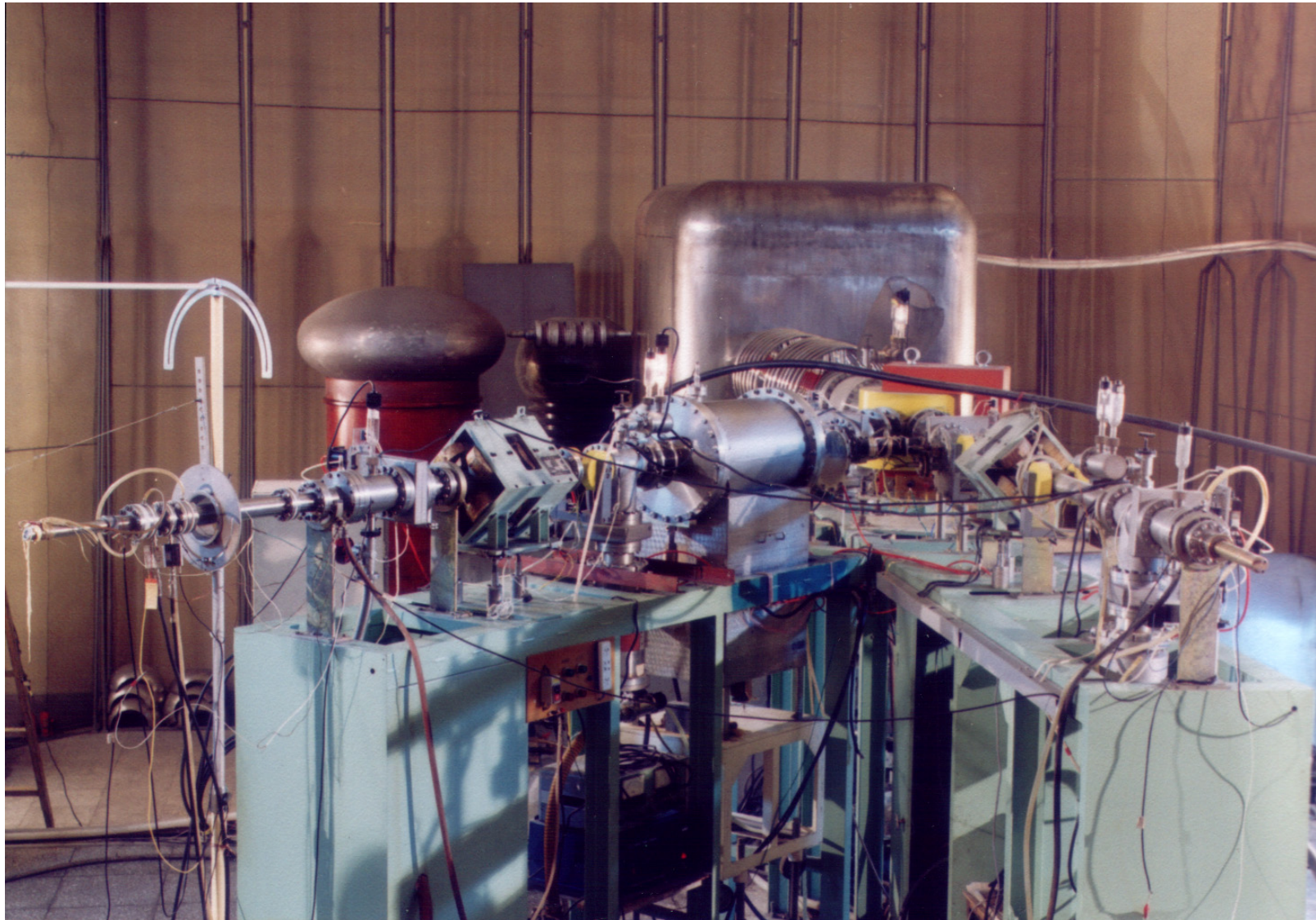
Facilities and Neutron Sources

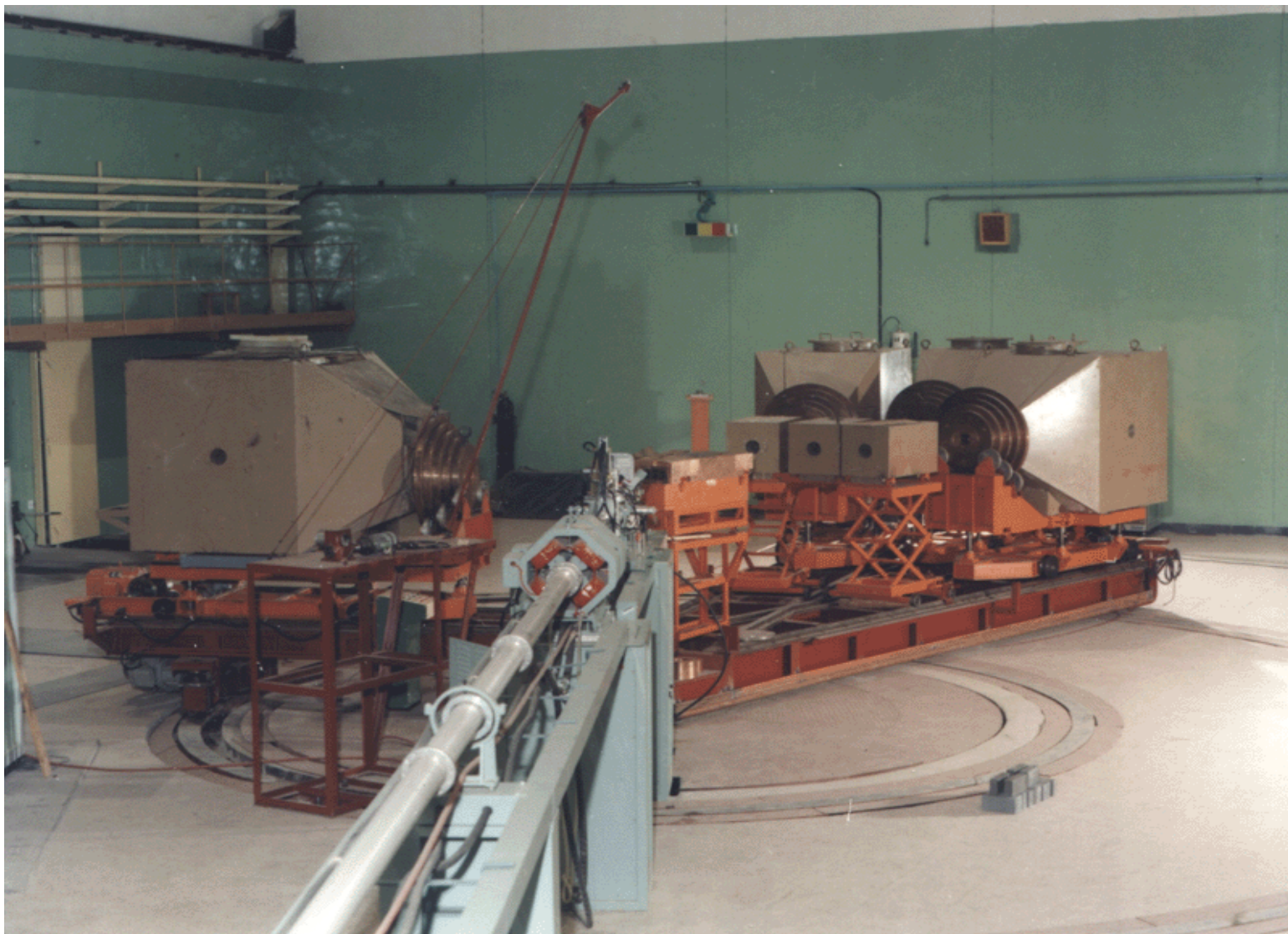
Facilities	Neutron Sources	Intensities(n/s)
Reactors		10^{14}
HI-13 (15MV x 2)	8-14 MeV (d+D) 4-10 MeV (p+T) 22-42 MeV (d+T)	10^9 10^8 10^7
1.7MV x 2	3-6 MeV (d+D) 14-20 MeV (d+T) 0.07-2.5 MeV (p+T) 0.03-1.7 MeV (p+Li)	10^{10} 10^9 10^{10} 10^9
Generator	2.5, 14 MeV (dc/Pulsed)	10^{11}

HI-13 Tandem



Neutron generator





Target and Detector System

- **TOF (HI-13)**

Flight path: 5-10 m

Three detectors: 4" x 2"; 7" x 4"

- **TOF (Generator)**

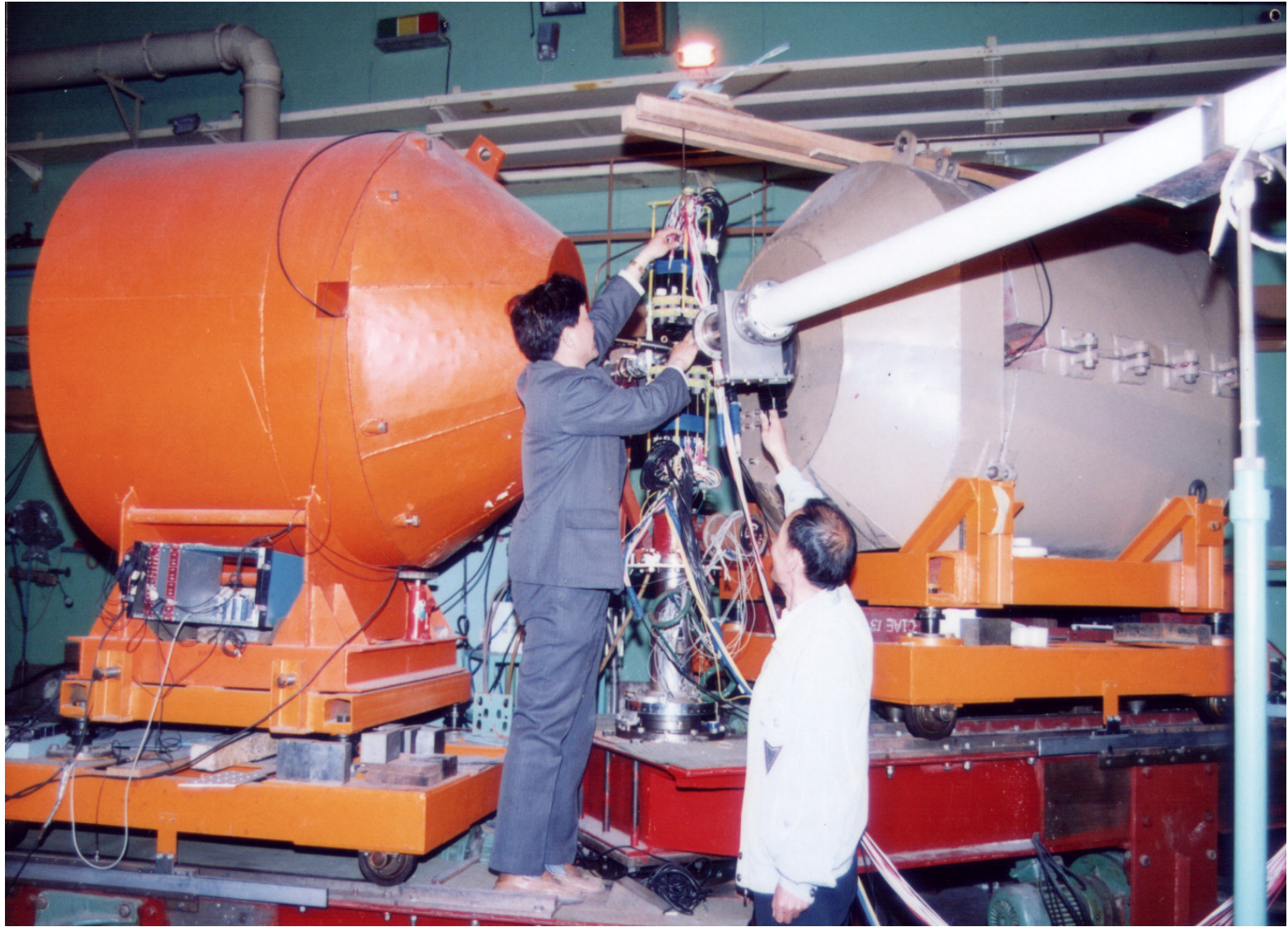
Flight path: 8 m

Three detectors: 7" x 2"

- **In beam g spectrometer**

2 NaI (10" x 10") + Plastic ring

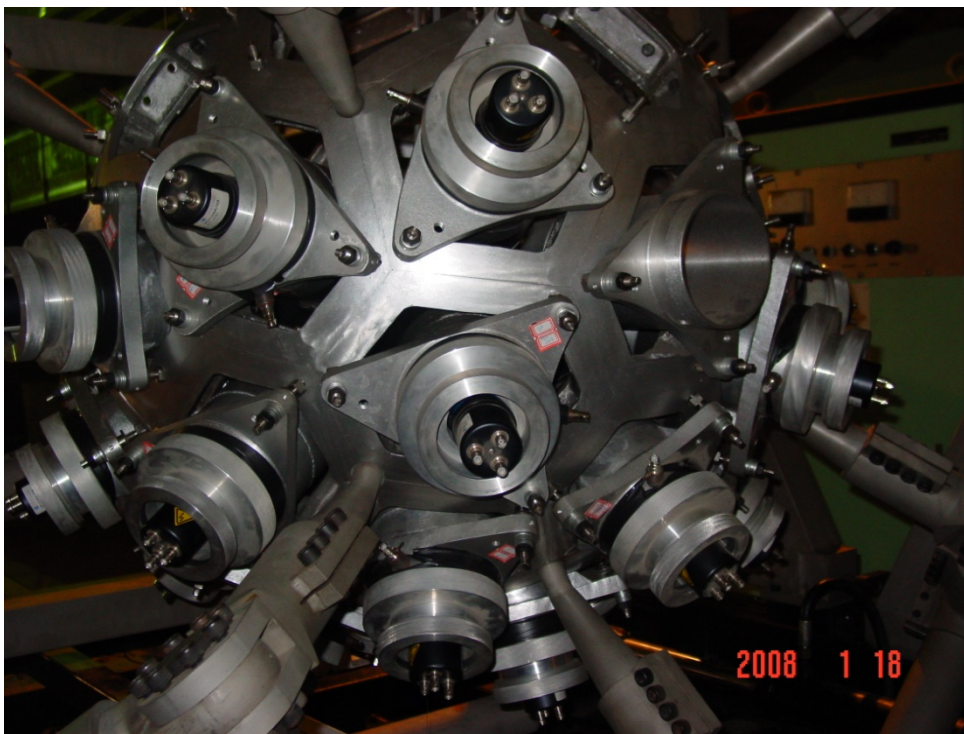
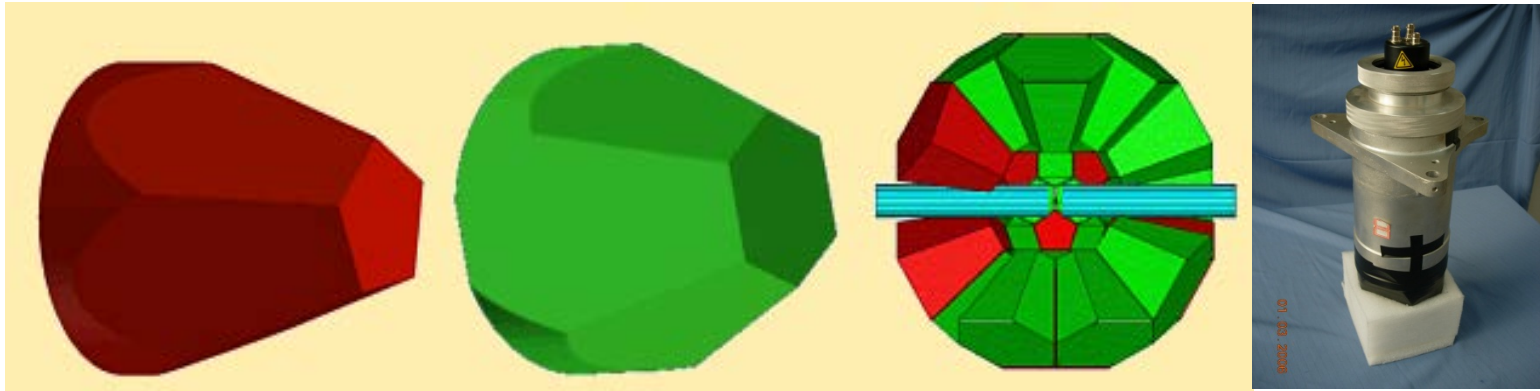
HPGe(60%) + BaF ring; BGO



In beam γ spectrometer



GTAF(Gamma Total Absorption Facility) detector in CIAE



γ production cross sections

Fe, Al, C, O, N, ^{238}U

Fission

Fission Prompt neutron spectra of ^{238}U

Fission fragments yields (Thermal-22 MeV)

Excitation Function

p, d, α induced activation cross sections

neutron induced cross sections

carefully done for low energy background

(d-D self build in; breakup, others)

Differential cross section for neutron scattering for ^{209}Bi at 37 MeV

Zuying Zhou

周祖英

Sep. 7, 2011

1. Introduction

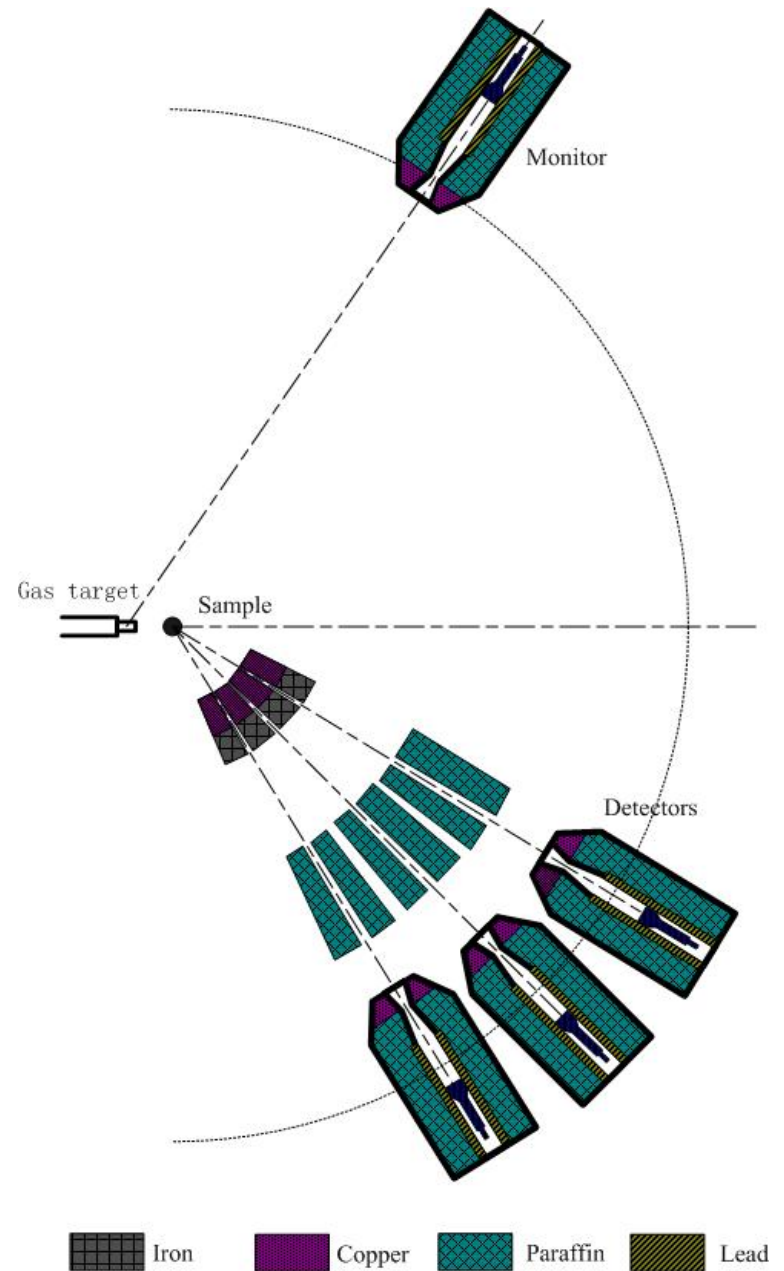
**Bi is one of the important materials for the design of a new generation of reactors
a possible solution for the spallation target, moderator, coolant of ADS**

- **^{209}Bi is a natural 100% isotopic pure**
- **The data are few at neutron energy over 20 MeV**
- **HI-13 tandem accelerator $E_{\text{max}} = 26 \text{ MeV}$,
then $E_{\text{n,max}} = 40 \text{ MeV}$ from $\text{T(d,n)}^4\text{He}$ source**

2. Experiment

• TOF technique

- * Flight path: ~ 6 m
3 main detectors
1 monitor
- * Angular distribution:
10-160 deg
- * Pulsed beam:
 $f = 4$ MHz,
Time width 2 ns,
Averaging current $I = 2.5$ μ A



Fast Neutron TOF Spectrometer

• Neutron Source

* Reaction ${}^3\text{H}(\text{d},\text{n}){}^4\text{He}$

* Tritium gas cell

Consisting of a T-cell
and a He-cell

Two Mo windows

One U oven

Two transducer

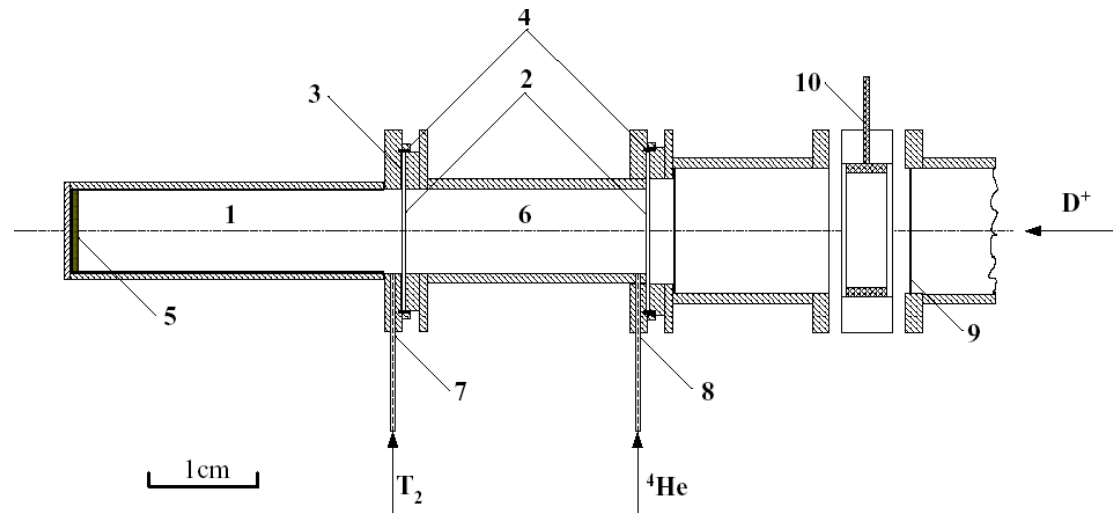
Ventilation glove box

Fast –acting valve

* Neutron energy

$E_d = 20 \text{ MeV}$

$E_n = 37 \text{ MeV}$



1- tritium gas cell 2- Mo foil 3- indium O-ring 4- rubber O-ring
5- gold beam stop 6- ${}^4\text{He}$ gas cell 7- tritium gas tube 8- ${}^4\text{He}$ gas tube
9- Ta diaphragm 10- electron suppressor

Neutron source: ${}^3\text{H}(\text{d},\text{n}){}^4\text{He}$

${}^3\text{H}$ gas cell: $\Phi 12 \times 46$, ${}^3\text{H}$: 2.2 atm

He gas cell: $\Phi 12 \times 22$, He: 0.2 atm

• Sample and geometry

* Metallic Bi samples:

$\Phi 15 \times 35$ ($< 28^\circ$)

$\Phi 25 \times 35$ ($> 28^\circ$)

* Geometry:

Neutron source ---sample: $l = 20$ cm

Sample – Detector: $d = \sim 6$ m

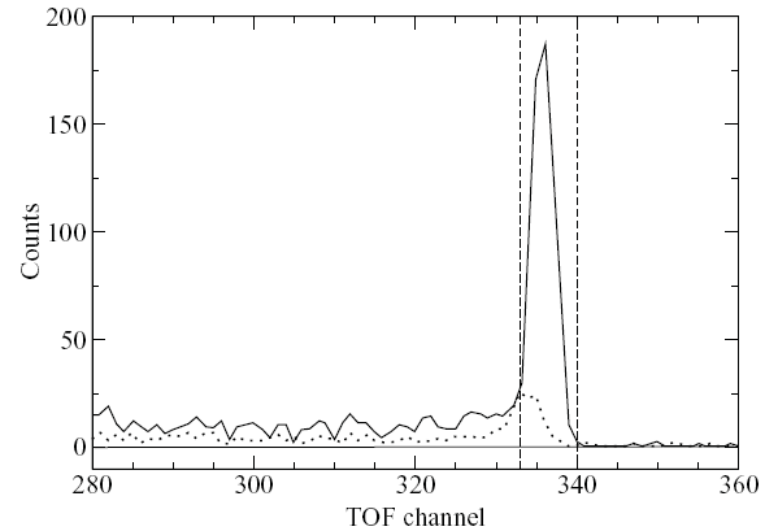
Angular distribution:

10-160 deg steps: 3-5deg

Total 43 angles

Detectors:

ST-451(NE-213 equ.) $\Phi 100 \times 50$



- Reference cross section: ${}^3\text{H}(\text{d},\text{n}){}^4\text{He}$ at 0 deg

The yield in detector: $\longrightarrow M(0^\circ) = Q n_T \sigma_{dT}(0^\circ) \frac{A \varepsilon_i}{(d + l)^2}$

(keep beam current as low as 100nA)

The neutron flux at sample: $\longrightarrow N(0^\circ) = Q \frac{n_T \bar{\sigma}_{dT}}{l^2}$
 small sample : -- $\pm 2.15^\circ$,
 big one -- $\pm 3.58^\circ$

Combining and getting: $\longrightarrow N(0^\circ) = M(0^\circ) \frac{\bar{\sigma}_{dT}}{\sigma_{dT}(0^\circ)} \frac{(d + l)^2}{l^2 A \varepsilon_i}$.

Then, for Bi : $\longrightarrow \sigma(\theta) = \frac{N(\theta)}{N(0^\circ)} \frac{d^2 l^2}{(d + l)^2} \frac{\varepsilon_i}{\varepsilon} \frac{1}{N_s}$.

Differential X-section of d-T reaction from DROSG-2000

- **Detector efficiency**

- **Experimental calibration for the relative efficiency**

$^2\text{H}(\text{d},\text{n})^3\text{He}$, $^3\text{H}(\text{d},\text{n})^4\text{He}$ reaction excitation functions at 0 deg and angular distribution were used to cover the neutron energy 8.7- 40.45 MeV.

- **Monte Carlo codes of SCINFUL and NEFF7 were used for the absolute efficiency**

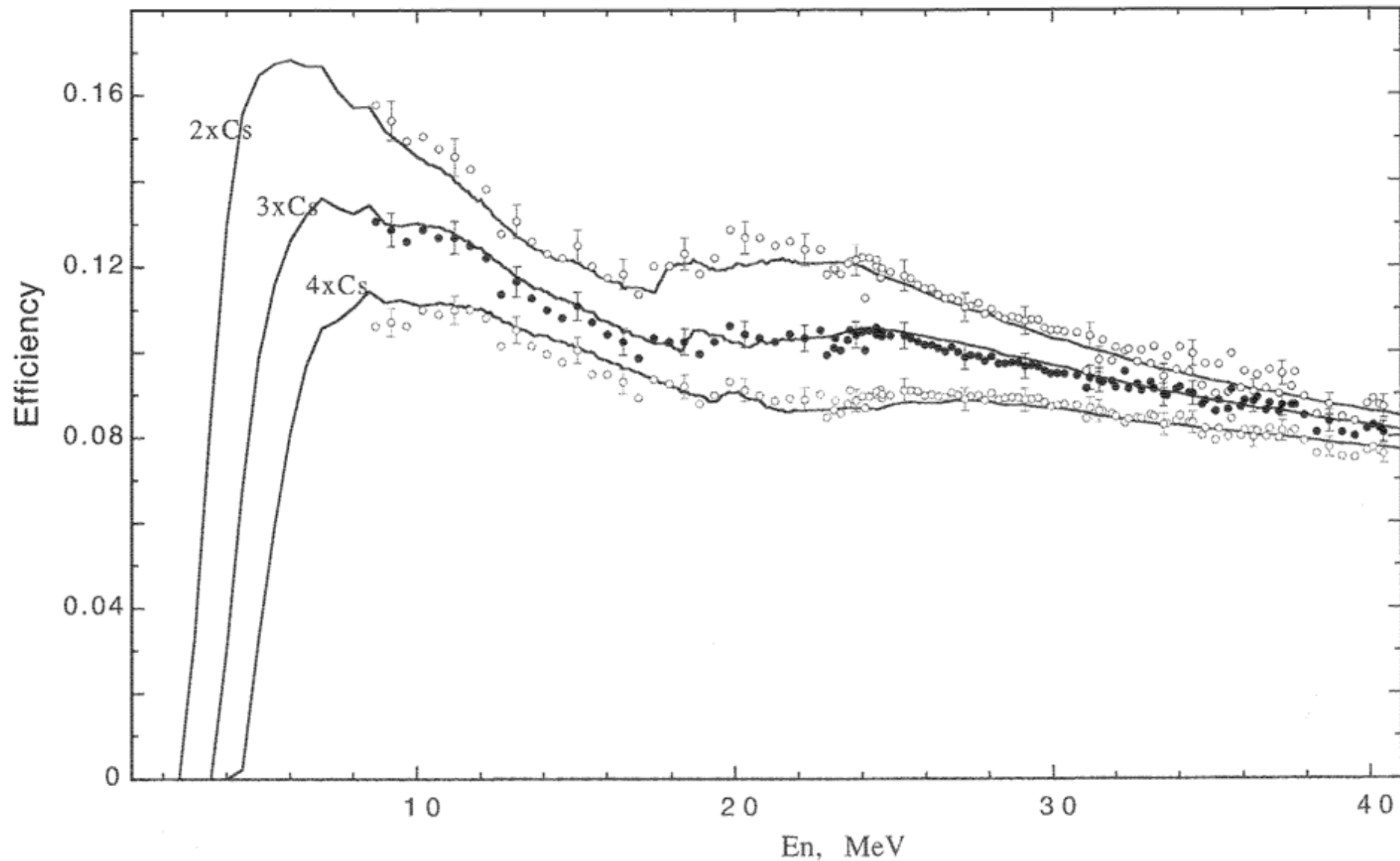
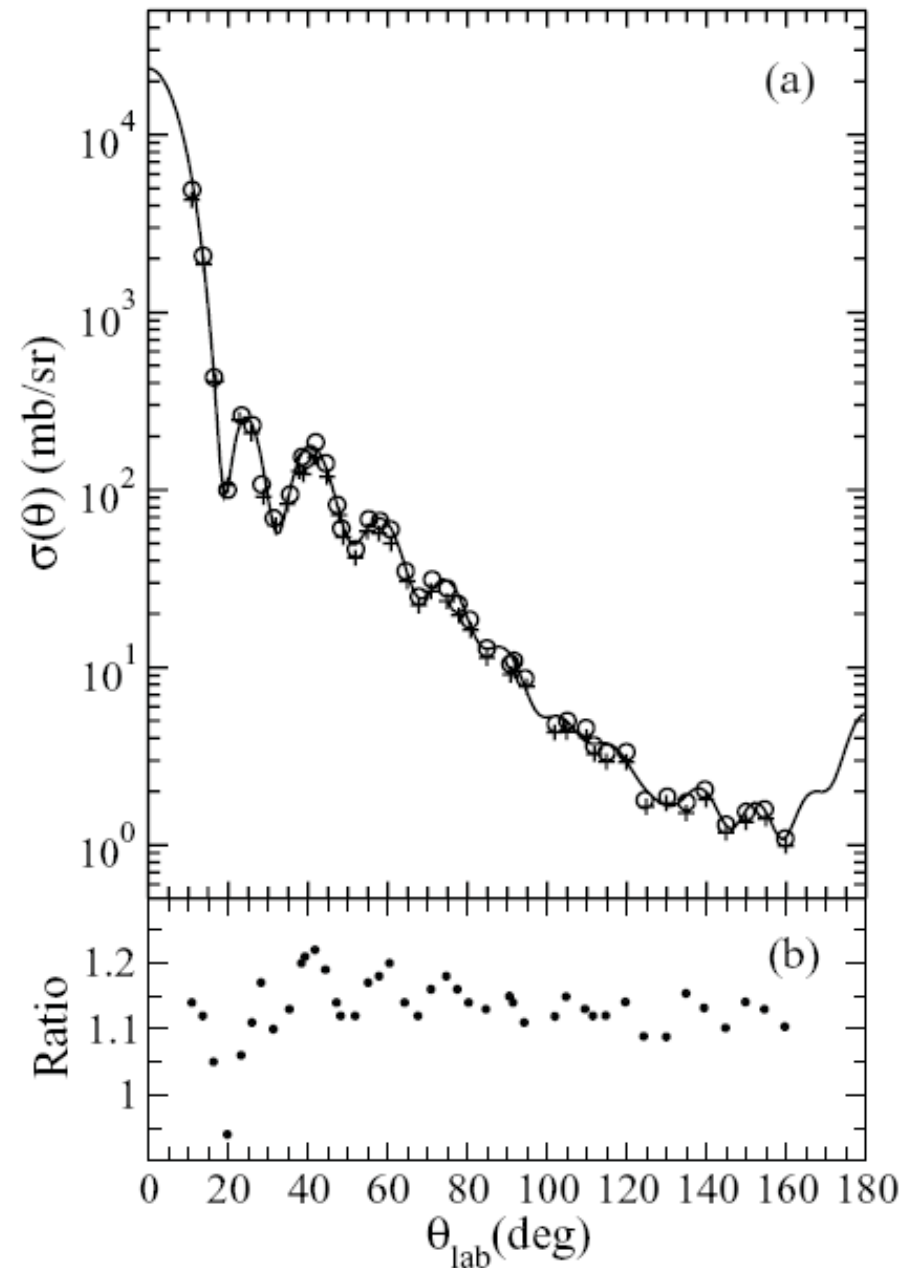


Fig. 2. Neutron detection efficiency of the organic scintillator for three biases. Curves: SCINFUL calculation; points: measurement. Error bar : 3%.

3. Data analysis

- * MC simulation for
(gas cell, sample, detectors)
Flux attenuation
Multiple-scattering
Finite geometry effects
- STREUER code (extended)
- Total x-s for Bi from JENDL/HE



•Model calculation

OM calculation shown lower than data between 60° - 120°



Inelastic scattering contamination ?

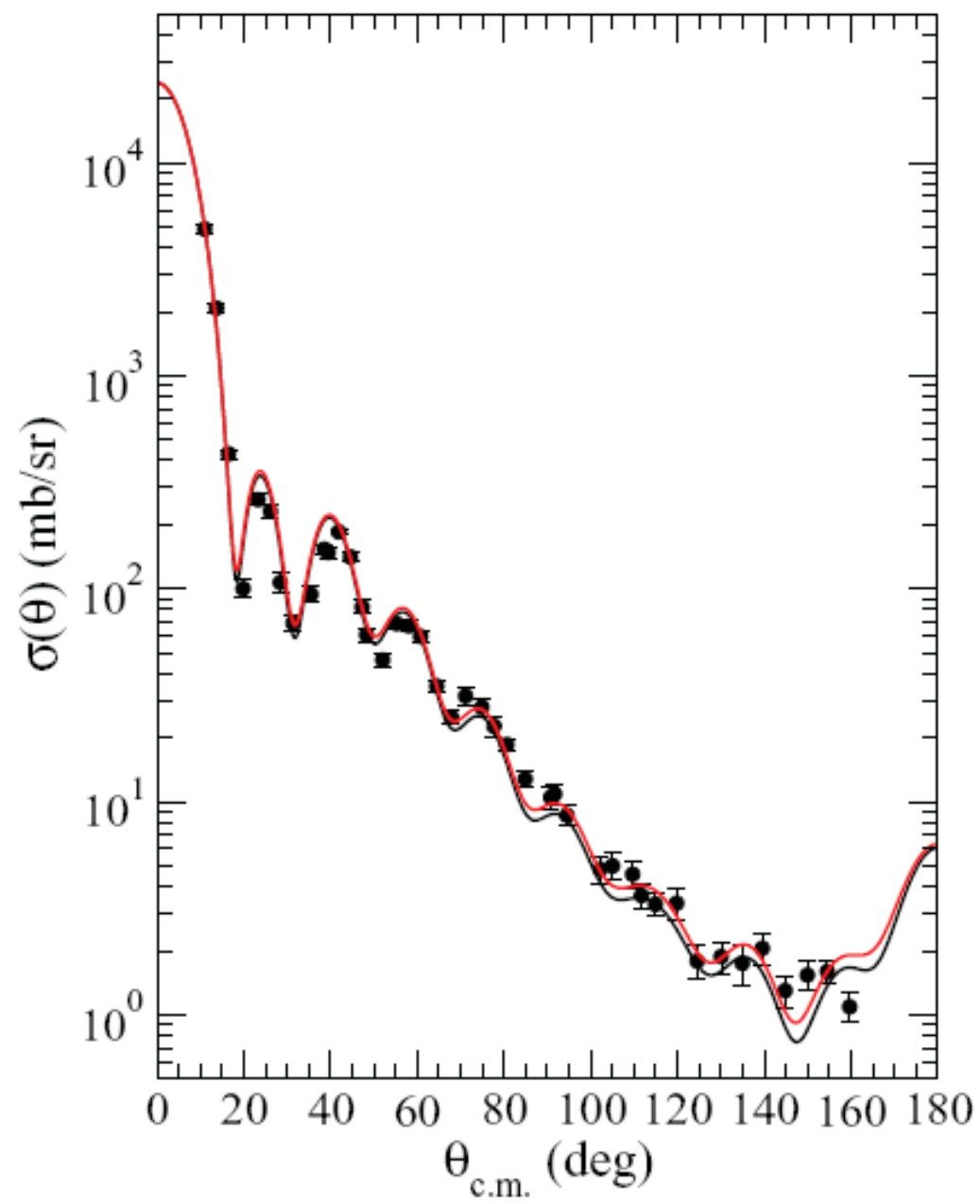
$$\Delta E = \sim 2.7 \text{ MeV}$$



Using couple channel code to get inelastic ones

Add them to those from OM calculation

The optical potentials from [A. J. Koning, and J. P. Delaroche, Nucl. Phys. A 713, 231 \(2003\).](#)



4 Conclusion

- 1, Total Cross sections and elastic differential cross sections over 20 MeV are quit important .**
- 2, The more accurate data are helpful to reduce the ambiguities of OMP parameters and improve the prediction and data evaluations.**

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Differential cross section for neutron scattering from ^{209}Bi at 37 MeV and the weak particle-core coupling

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Thank You!